

Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385



MAY 18 2006

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Serial No.	06-002A
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Docket No.	50-423
License No.	NPF-49

DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 3
LICENSEE EVENT REPORT 2005-005-01,
AUTOMATIC REACTOR TRIP OF MILLSTONE
UNIT 3 DUE TO LOW-LOW STEAM GENERATOR LEVEL

This letter forwards Licensee Event Report (LER) 2005-005-01. This is revision 1 to the LER 2005-005-00, which documented an event that occurred at Millstone Power Station Unit 3, on December 1, 2005. Revision 0 of the LER was submitted pursuant to 10 CFR 50.73(a)(2)(iv)(A), as an event that resulted in manual or automatic actuation of systems listed in 50.73(a)(2)(iv)(B).

If you have any questions or require additional information, please contact Mr. David W. Dodson at (860) 447-1791, extension 2346.

Very truly yours,


J. Alan Price
Site Vice President - Millstone

TE22

Attachments: 1

Commitments made in this letter: None.

cc: U.S. Nuclear Regulatory Commission
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Mr. S. M. Schneider
NRC Senior Resident Inspector
Millstone Power Station

Attachment 1

Licensee Event Report 2005-005-01,
Automatic Reactor Trip of Millstone Unit 3 Due to Low-Low Steam Generator Level

Millstone Power Station Unit 3
Dominion Nuclear Connecticut, Inc. (DNC)

NRC FORM 366 (6-2004)		NUCLEAR REGULATORY COMMISSION		APPROVED BY OMB: NO. 3150-0104 Estimated burden per response to comply with this mandatory information collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0066), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.		EXPIRES: 06/30/2007			
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)									
FACILITY NAME (1) Millstone Power Station – Unit 3				DOCKET NUMBER (2) 05000423		PAGE (3) 1 of 2			
TITLE (4) Automatic Reactor Trip of Millstone Unit 3 Due to Low-Low Steam Generator Level									
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)		OTHER FACILITIES INVOLVED (8)	
MO	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MO	DAY	YEAR	FACILITY NAME
12	01	2005	2005-005-01			05	18	2006	DOCKET NUMBER
OPERATING MODE (9)		1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply) (11)						
POWER LEVEL (10)		038	20.2201(b)		20.2203(a)(3)(ii)		50.73(a)(2)(ii)(B)		50.73(a)(2)(ix)(A)
			20.2201(d)		20.2203(a)(4)		50.73(a)(2)(iii)		50.73(a)(2)(x)
			20.2203(a)(1)		50.36(c)(1)(i)(A)		X 50.73(a)(2)(iv)(A)		73.71(a)(4)
			20.2203(a)(2)(i)		50.36(c)(1)(ii)(A)		50.73(a)(2)(v)(A)		73.71(a)(5)
			20.2203(a)(2)(ii)		50.36(c)(2)		50.73(a)(2)(v)(B)		OTHER
			20.2203(a)(2)(iii)		50.46(a)(3)(ii)		50.73(a)(2)(v)(C)		Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iv)		50.73(a)(2)(i)(A)		50.73(a)(2)(v)(D)		
			20.2203(a)(2)(v)		50.73(a)(2)(i)(B)		50.73(a)(2)(vii)		
			20.2203(a)(2)(vi)		50.73(a)(2)(i)(C)		50.73(a)(2)(viii)(A)		
			20.2203(a)(3)(i)		50.73(a)(2)(ii)(A)		50.73(a)(2)(viii)(B)		
LICENSEE CONTACT FOR THIS LER (12)									
NAME David W. Dodson, Supervisor Nuclear Station Licensing						TELEPHONE NUMBER (Include Area Code) (860) 447-1791			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION		MONTH	DAY
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE).						<input checked="" type="checkbox"/> NO		DATE (15)	
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)									
<p>At approximately 14:45 on December 1, 2005, with Millstone Unit 3 (MP3) in Mode 1 at an indicated power level of 38%, an automatic reactor trip occurred on 'C' Steam Generator Low-Low level (S/G Low-Low level). At the time of the reactor trip, a downpower to 30% was in progress to facilitate a containment entry to locate and repair a Reactor Coolant System (RCS) leak. As turbine load and reactor power were reduced, main turbine vibrations increased above allowable limits and the main turbine was manually tripped in accordance with procedure. After the turbine was manually tripped, the reactor automatically tripped on 'C' S/G Low-Low level. All safety systems performed as designed, including auto actuation of the auxiliary feedwater system. The balance of the plant shutdown was uncomplicated.</p> <p>The direct cause of the reactor trip was a Low-Low level in the 'C' S/G that resulted from the shrink in S/G water level created by the manual turbine trip. The plant is designed to maintain the reactor critical following a turbine trip below 45% power. At the time of the trip, the Reactor Protection System S/G Low-Low Level Trip setpoints were being maintained at 27% vice the nominal value of 18% to address Westinghouse identified S/G level inaccuracies associated with the S/G mid-deck plate differential pressure. Operation at the increased set point reduced the margin available to accommodate S/G level transients that occur following a turbine trip at power. The automatic reactor trip and auxiliary feedwater system actuation are reportable in accordance with 10CFR50.73(a)(2)(iv).</p>									

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
Millstone Power Station – Unit 3	05000423	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 2
		2005	005 --	01	

NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

1. Event Description

At approximately 14:45 on December 1, 2005, with Millstone Unit 3 (MP3) in Mode 1 at an indicated power level of 38%, an automatic reactor trip occurred on 'C' Steam Generator Low-Low level (S/G Low-Low level). At the time of the reactor trip, a power reduction to 30% was in progress to facilitate a containment entry to locate and repair a Reactor Coolant System (RCS) [AB] leak. As turbine [TA] load and reactor power were reduced, main turbine vibrations increased above allowable values and the main turbine was manually tripped in accordance with procedure. After the turbine was manually tripped, the reactor automatically tripped on 'C' S/G Low-Low level. All safety systems performed as designed, including auto actuation of the auxiliary feedwater system. The balance of the plant shutdown was uncomplicated.

The automatic reactor trip is reportable as a reactor protection system [JC] actuation in accordance with 10CFR50.73(a)(2)(iv). The actuation of the auxiliary feedwater system [BA] [JD] is also reportable in accordance with 10CFR50.73(a)(2)(iv)A.

2. Cause

The direct cause of the reactor trip was a Low-Low level in the 'C' S/G that resulted from the shrink in S/G water level created by the manual turbine trip. The plant is designed to maintain the reactor critical following a turbine trip below 45% power (P-9 permissive setpoint). At the time of the trip, the Reactor Protection System (RPS) S/G Low-Low Level Trip setpoints were being maintained at 27% vice the nominal value of 18% to address Westinghouse identified S/G level inaccuracies associated with the S/G mid-deck plate differential pressure. Operation at the increased set point reduced the margin available to accommodate S/G level transients that occur following a turbine trip at power. During normal plant shutdowns, the Low-Low S/G Level setpoint is manually reduced to 18% at power levels less than 40% rated thermal power. This restores the margin to trip to the design value at low power levels. During rapid power reductions there is insufficient time to change the narrow range level setpoint.

3. Assessment of Safety Consequences

The reactor trip occurred at a conservatively high S/G level, was uncomplicated, and all safety functions were accomplished per design. A reactor trip on Low-Low S/G level protects the reactor from a loss of heat sink.

4. Corrective Action

An engineering analysis to determine the new S/G Low-Low setpoints to provide additional margin is being evaluated. Additional corrective actions are being taken in accordance with the station's corrective action program.

5. Previous Occurrences

There have been no occurrences of a reactor trip following a turbine trip below the P-9 permissive setpoint in the previous 3 years.

Energy Industry Identification System (EIIIS) codes are identified in the text as [XX].